

## Changes to Delayed Neutron Measurements at RMCC and their corresponding MCNP6 Simulations

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### 1. Experimental Data Sets

Solutions were prepared from CRM standards and further diluted with nitric acid and distilled water. Samples were placed in polyethylene vials before transport to an inner SLOWPOKE-2 irradiation site where they were exposed to a predominately thermal neutron flux for 60 s. After irradiation the samples were sent to an array of  $^3\text{He}$  detectors which recorded the DN emissions as a function of count time up to 3 minutes. Further details regarding the delayed neutron counting system and these measurements can be found in references 1 and 2 respectively. Experimental data has been corrected for dead time effects and neutron background contributions [3]. **Measurements have been normalized by fissile mass [g] and detection efficiency (33 %) to obtain DN emission rate,  $Q(t)$  [ $\text{s}^{-1}\text{g}^{-1}$ ].** Each isotope ( $^{233}\text{U}$ ,  $^{235}\text{U}$  and  $^{239}\text{Pu}$ ) was irradiated and counted in triplicate, the provided measurements represent their average  $Q(t)$ .

### 2. MCNP Simulations

Atomic Energy of Canada Limited has provided a MCNP input deck containing LEU SLOWPOKE-2 dimension and material specifications, the contents of which are detailed in Ref. 4. This input deck was modified to include a polyethylene vial within an inner irradiation site to determine a higher fidelity neutron flux spectrum. This flux was recreated within the vial solution of a second input deck, which includes the irradiation of a fissile solution for 60 s and the recording of subsequent DN emissions from the vial. The DN emission Rate,  $Q(t)$  [ $\text{s}^{-1}\text{g}^{-1}$ ] was compared to the normalized measurements described in the previous section. The final deck simulated energy deposition within the  $^3\text{He}$  detectors, Figure 4. MCNP6v1 was used in the following simulations (load date: 05/08/13).

### 3. Comparisons

Measurements and simulations are now compared as DN emission rates,  $Q(t)$  [ $s^{-1}g^{-1}$ ]. Examples are shown in Figures 1-3. This work has been submitted for publication in the Journal of Nuclear Technology.

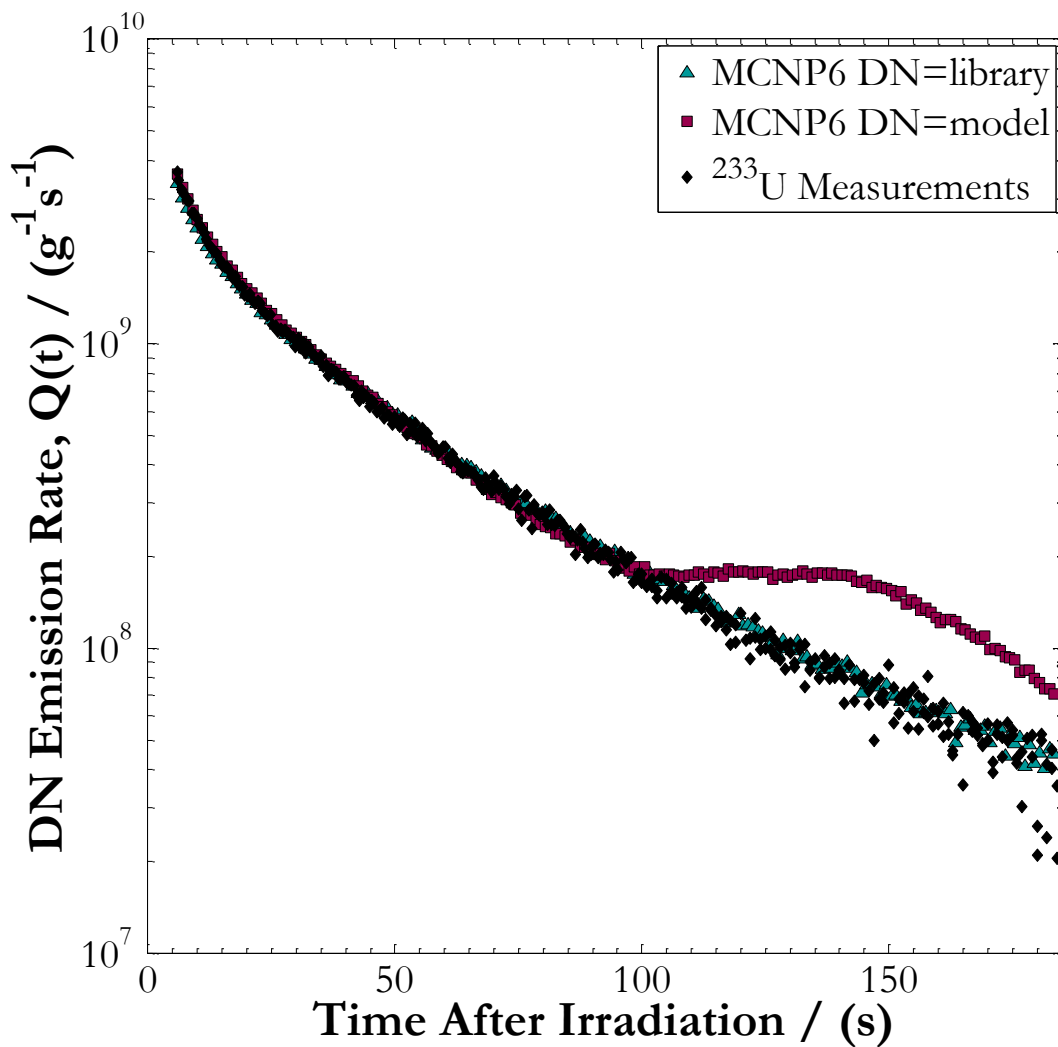


Figure 1: DN Emission Rates from U-233

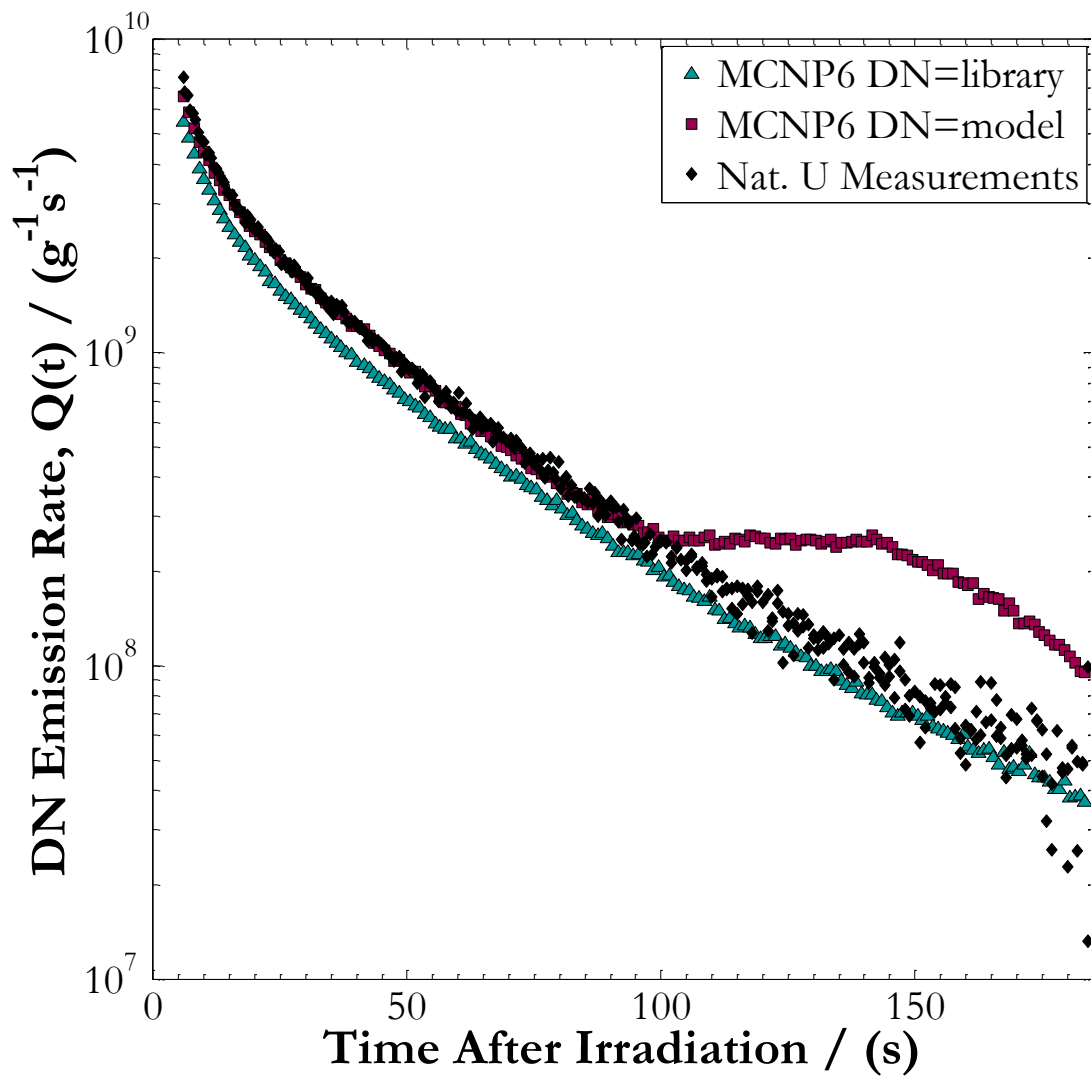


Figure 2: DN Emission Rates from Nat U.

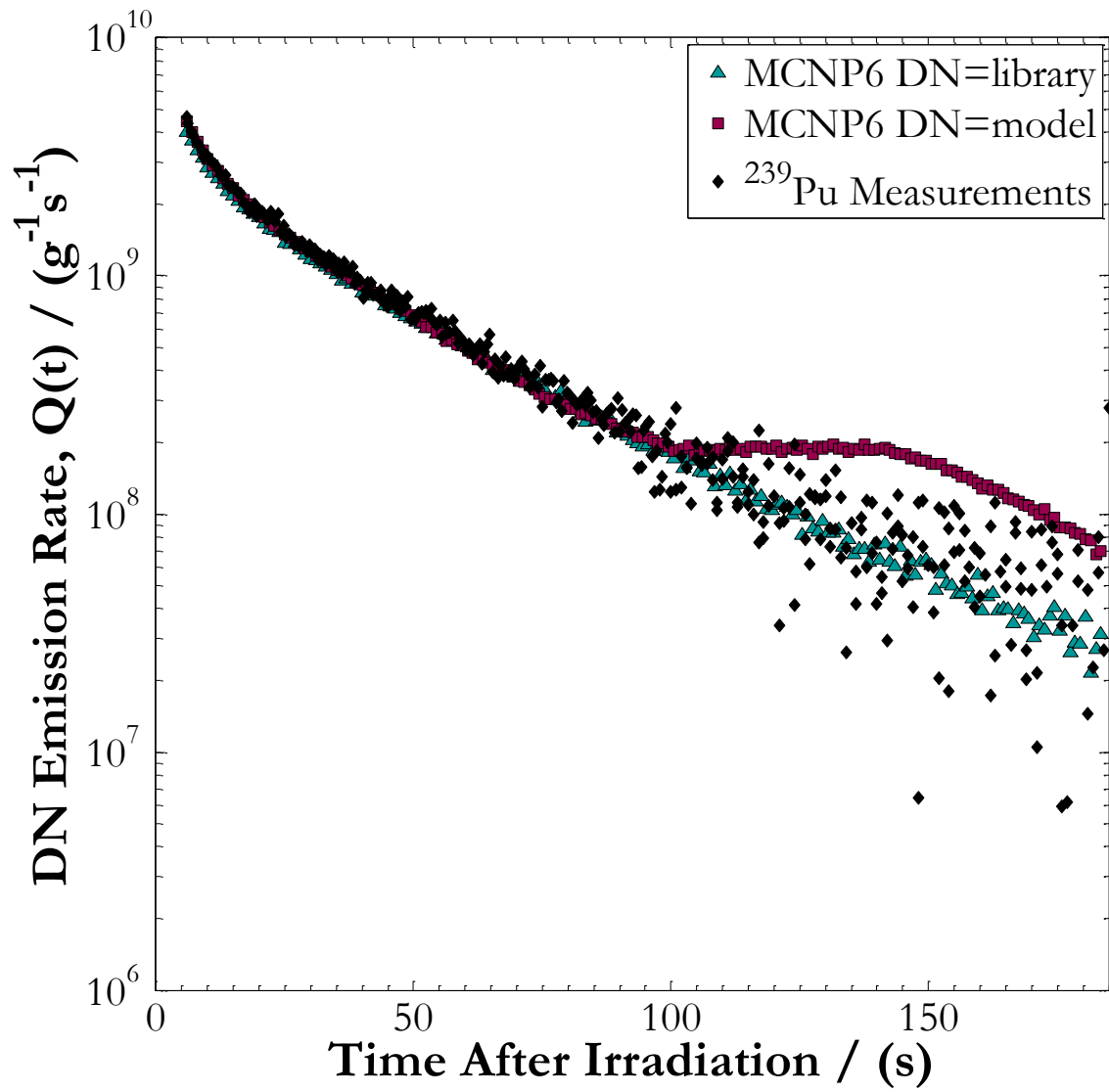


Figure 3: DN Emission Rates from Pu-239.

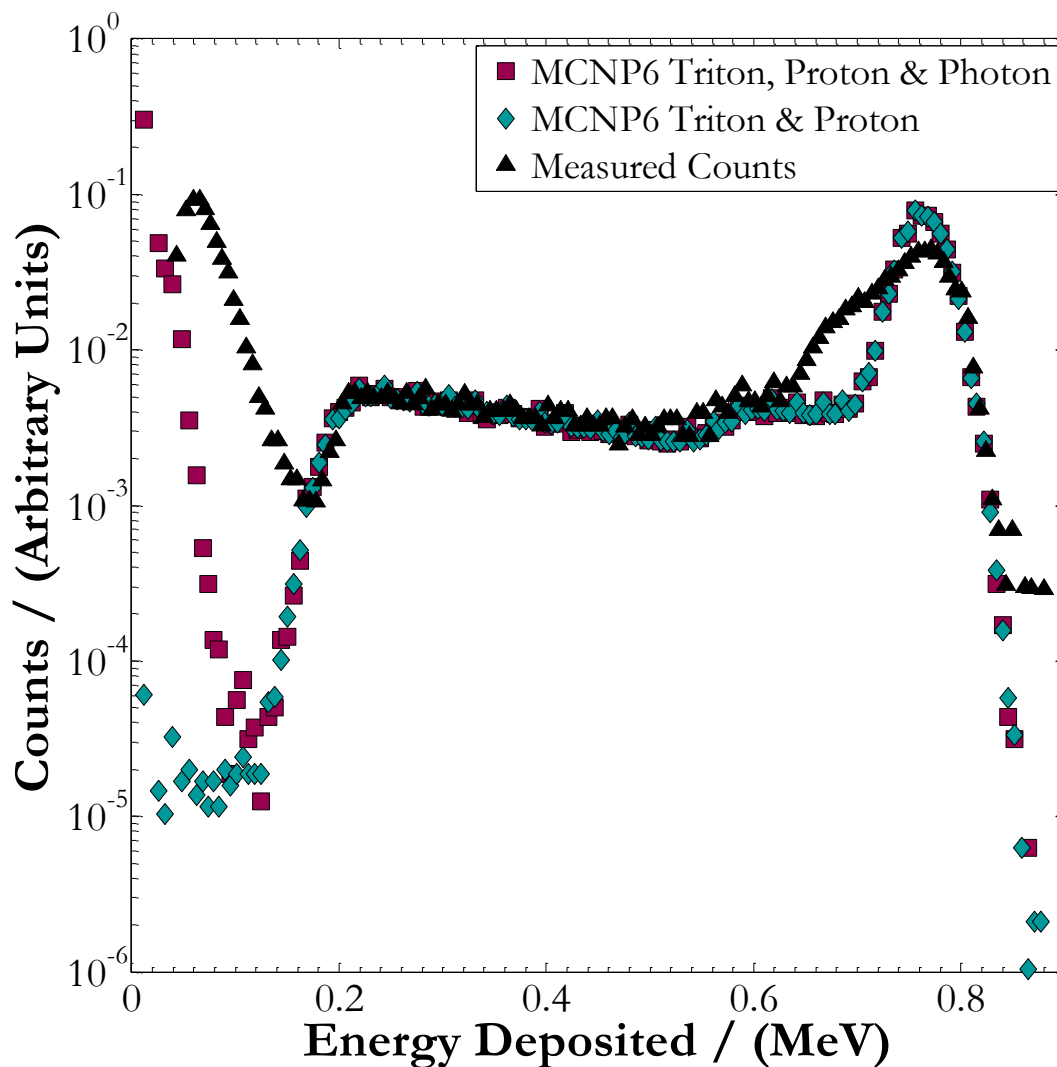


Figure 4: Energy Deposition in <sup>3</sup>He Detectors

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## References

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1. M.T. Sellers, D.G Kelly, and E.C. Corcoran, "An automated delayed neutron counting system for the mass determination of special nuclear materials" *Radioanalytical and Nuclear Chemistry* **291** 2 (2012) 281 - 285.
2. M.T. Sellers, J.T. Goorley, E.C. Corcoran, D.G. Kelly. "Modeling the detection of delayed neutron signatures in MCNP6 and comparisons with experimental  $^{233}\text{U}$ ,  $^{235}\text{U}$  and  $^{239}\text{Pu}$  Measurements" LA-UR-13-22525, Los Alamos National Laboratory Report, (2013).
3. M.T. Sellers, E.C. Corcoran, D.G. Kelly, "The analysis and attribution of the time-dependent neutron background resultant from sample irradiation in a SLOWPOKE-2 reactor" *Radioanalytical and Nuclear Chemistry* **295** 2 (2013) 1221-1228.
4. T.S.Nguyen, G.B. Wilkin, and J.E Atfield "Monte Carlo Calculations Applied to SLOWPOKE Full Reactor Analysis" *AECL Nuclear Review*, **1** 2 (2012) 43-46.